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ADAPTATIONS OF A NUCLEAR REACTOR MODEL TOWARDS MORE FLEXIBILITY in order to accommodate a power system with a high insertion of variable renewable energy sources

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CEA CONTEXT



Power system:

- Role: transport electricity from generating units to load locations
- Objective: ensure production and consumption balance at all time
- Historically: imbalance on the load side only



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Actual and upcoming challenges:

- Demand side: Continuous variation of the load BUT demand management, new uses, ...
- Supply side: RES* ↑ and high CO2 emitting power plant → 0 BUT electric mobility park, decentralised productions...

*RES: Renewable Energy Sources

Cea context

- In case of disturbance:
 - Caused by load or generation variation, grid default, ...
 - Frequency controls occur





CEA CONTEXT

In case of disturbance:

- Caused by load or generation variation, grid default, ...
- Frequency controls occur
- Levers such as NPP or other dispatchable units

Hydro	> 25
Diesel	25
Gas	7
Coal	5
Nuclear	5

%Pn/min max

* Mazauric et al, EPJN, under review

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- Nuclear in the frequency control:
 - Mainly used for load following because of the French fleet
 - Maximal power ramp of 5%Pn/min
- →This work is part of an overall methodology approach capable of defining a criterion at the interface of power systems and nuclear design*

→Minimal electric power ramp to ensure stability for high RES disturbance (%Pn/min): INPUT of this study*

STATE OF THE NUCLEAR HELP TO THE FREQUENCY CONTROL *Free dynamic*



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STATE OF THE NUCLEAR HELP TO THE FREQUENCY CONTROL *Primary frequency control*



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STATE OF THE NUCLEAR HELP TO THE FREQUENCY CONTROL *Load following*



OBJECTIVES OF THE STUDY

Goal:

- Build a simple NPP model capable of reproducing frequency transients as much as possible
- \circ Deduce ways to study more flexibility in the design of the reactor model
- ightarrow Observe the behaviour of a reactor and the analysis of variables of interest during power transients imposed by the electrical network.
- Modelling must be easily editable

Assumptions:

- Normal operation and close to the nominal operating point (100%Pn)
- One simplified regulation is taken into account (instead of 2), therefore only transients described as follow will be considered:
 - Electric power ramps greater than 5%Pn/min (maximal nuclear power ramp)
 - Larger variation magnitude than 2,5%Pn (current primary frequency magnitude)

Cea MODEL DESCRIPTION

- Model of a 1300 MW PWR
- Limit condition : SG power linked directly to electric

Neutronic point kinetic Enthalpy balances Thermal inertia



Simplified regulation



- With C-PWR-1300 from Corys : academic simulator
- High magnitude transient // load following : -10%Pn with 5%Pn/min maximal power ramp



Free dynamic is also validated

VALIDITY OF THE MODEL



CONSTRAINT-BASED MODEL

Higher ramp as input : -10%Pn/min and compared to the reference case -5%Pn/min



- Core response is accelarated
- But some safety parameters may be degraded because gradients are stronger

Cea SENSITIVITY STUDY - CARACTERISTICS

- Objectives of the study:
 - \circ Modifiy some parameters of the design
 - \circ Quantify the impact on the model thanks to indicators
- Parameters:
 - Neutronic
 - o Thermohydraulic
 - o Control

 \rightarrow Time delay of the entire primary loop τ

 Indicators: safety, performance and control use (normalization is done)

Factors	Unit	Factor's definition	Impacted domain
$\frac{\Delta}{\Delta t} P_{coeur}$	MW/s	Core power gradient averaged over 1 sec	Core performance
τ _{coeur}	S	Response time at 95% of core power i.e. time after which the core power has reached 95% of its final value	Core performance
$\frac{\Delta}{\Delta t}T_c$	°C/s	Fuel temperature gradient averaged over 1 sec	Safety of the core
$\frac{\Delta}{\Delta t}T_{e_{GV}}$	°C/s	Core inlet temperature gradient averaged over 1 sec	Safety of the steam generator
$\frac{\Delta}{\Delta t}T_{e_{coeur}}$	°C/s	Steam generator inlet temperature gradient averaged over 1 sec	Control rod use
$\Delta \rho_{ext}$	рст	Maximum amplitude of reactivity inserted or withdrawn by the power control	Control rod use and associated safety

SENSITIVITY STUDY – REFERENCE MODELS



SENSITIVITY STUDY – SOME RESULTS C27



Building of a simple PWR model capable of simulating frequency control transients driven by the input data %Pn/min (from power system)

- Possibility to modify some design parameters (example of τ)
- These parameters have impact on safety, flexibility, control use
- Part of a global approach at the interface of power system & nuclear design
- Perspectives :
 - Sensitivity of different parameters simultaneously in order to find a better compromise between perfomance / safety / control use
 Towards the use of this model for other core designs
 Coupling of the model with electric power system dynamic simulation software
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CONCLUSIONS



THANK YOU FOR YOUR ATTENTION !

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